

東京都市大学/早稲田大学 大学院共同原子力専攻主催

第10回未来エネルギーシンポジウム

－ 次世代原子炉の安全性はどこまで高められるのか？

2014年10月31日(金): 渋谷エクセルホテル東急(プラネッツルーム)

小型炉、革新的原子炉の 安全性

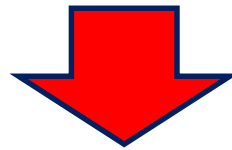
関本 博

都市大客員教授

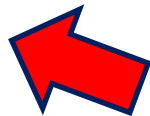
東工大名誉教授

福島第一原子炉事故

廃炉



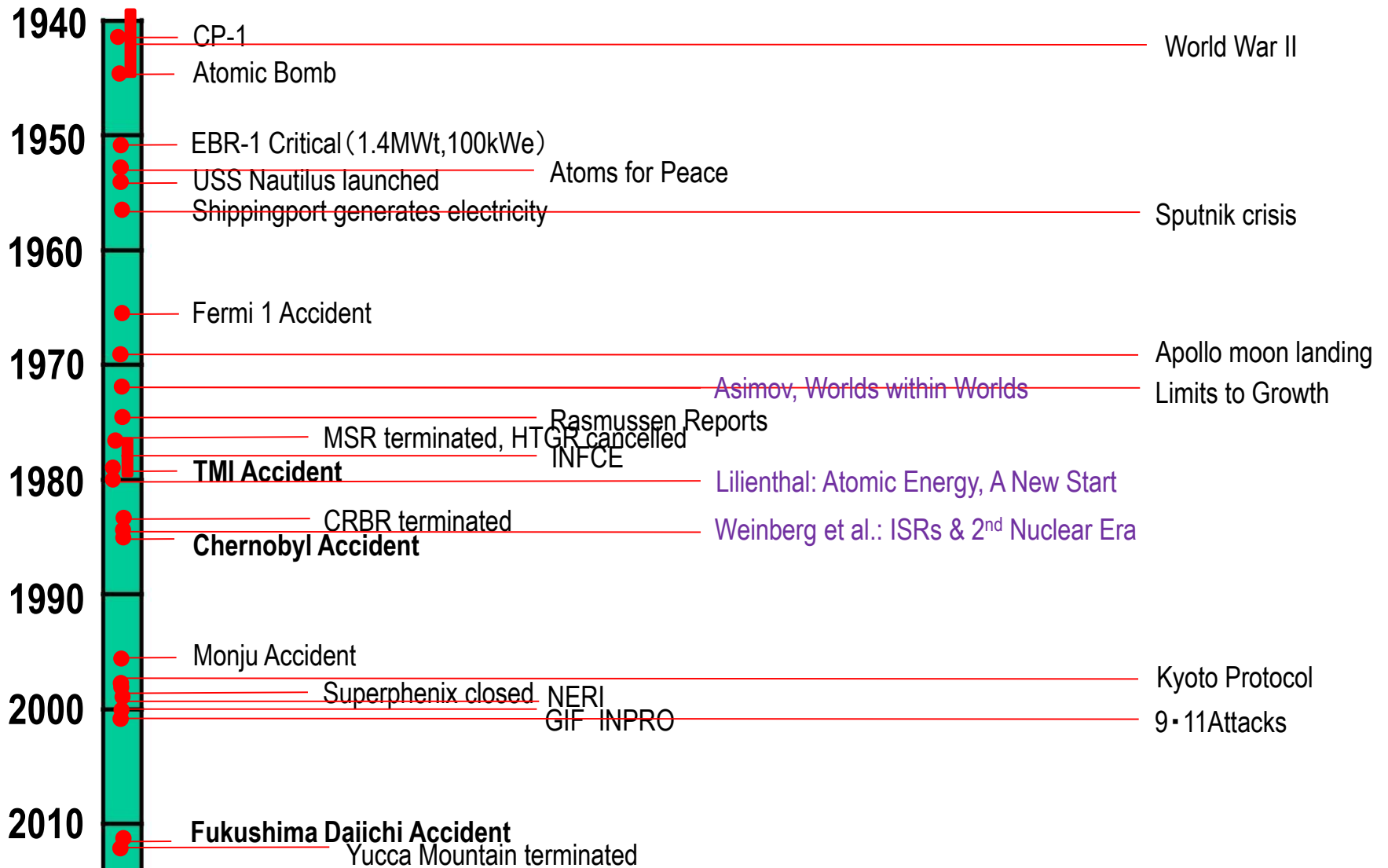
対応

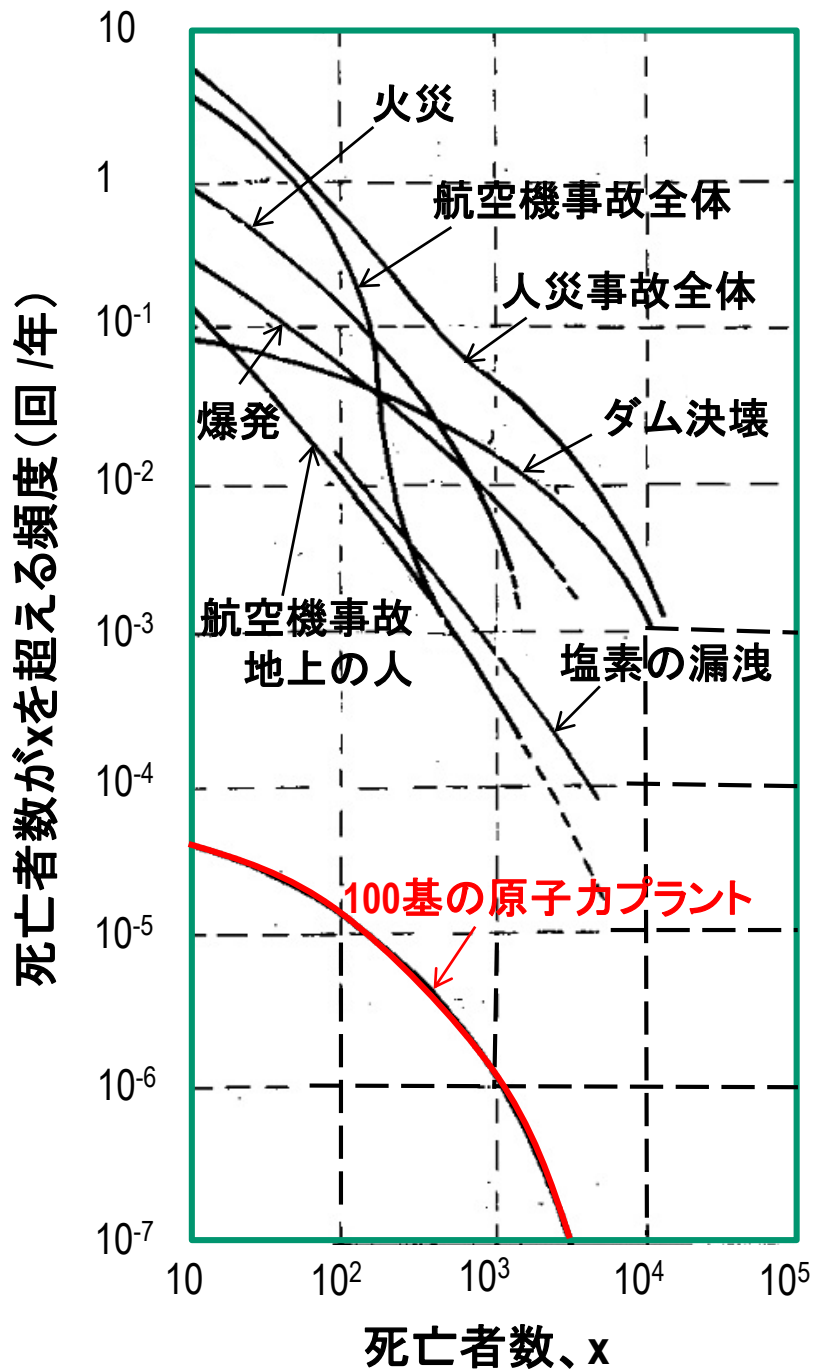


軽水炉運転再開

安全性を高める
ストレステスト
安全設備、設計変更
システム改善

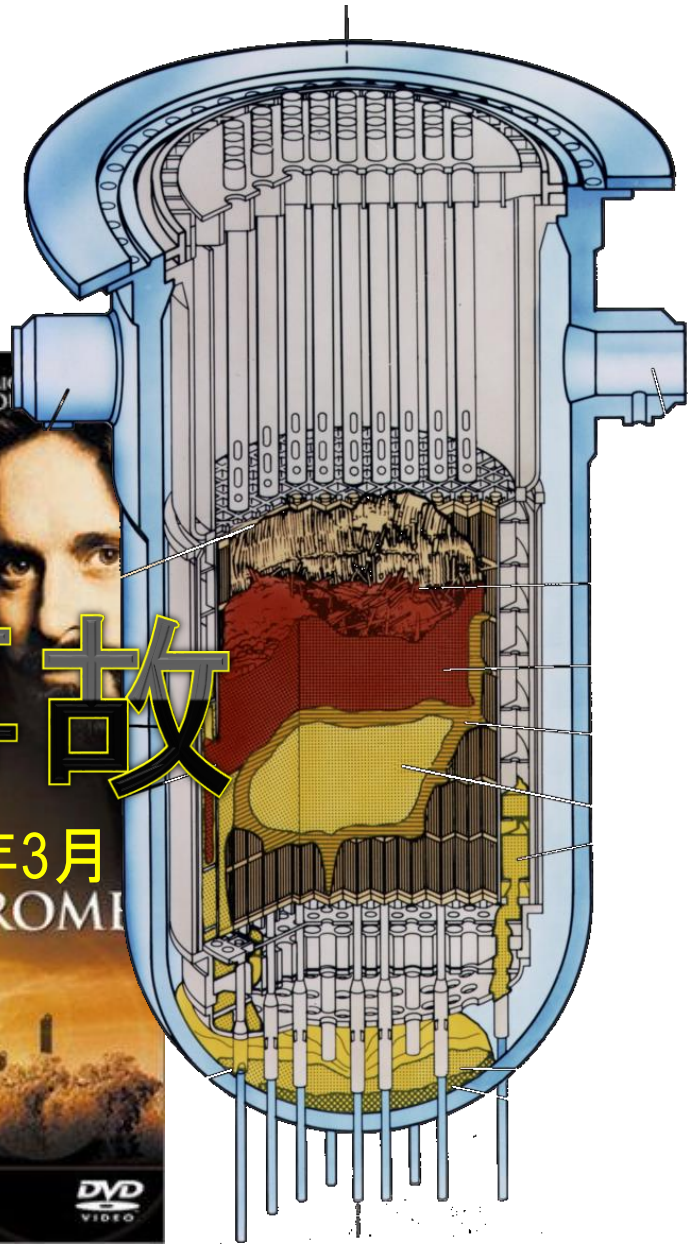
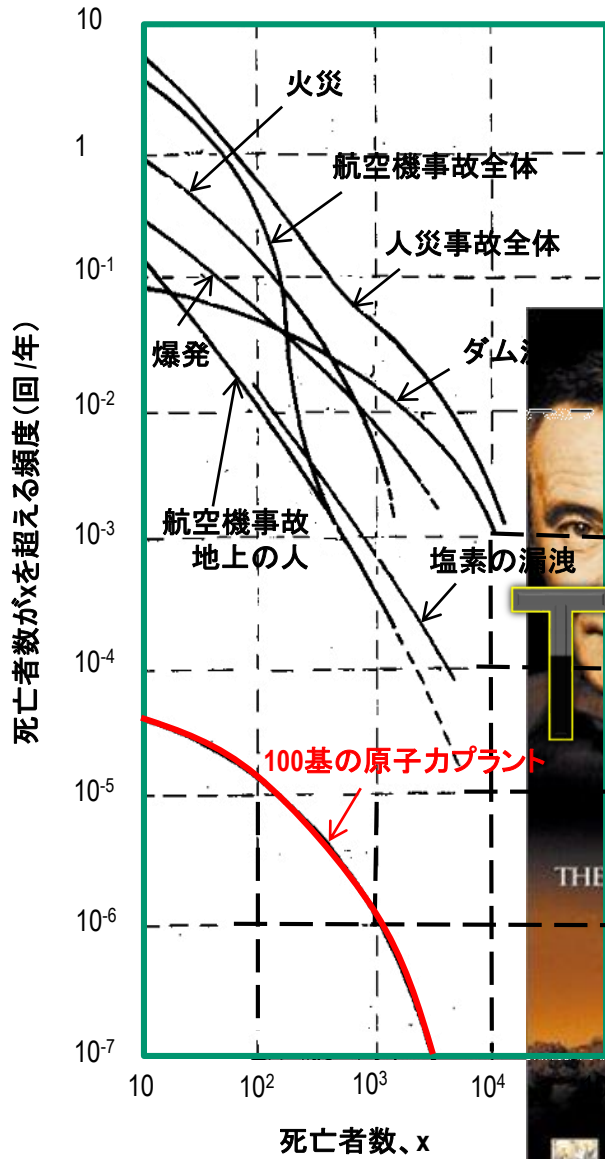
革新的原子炉に関連した簡単な歴史



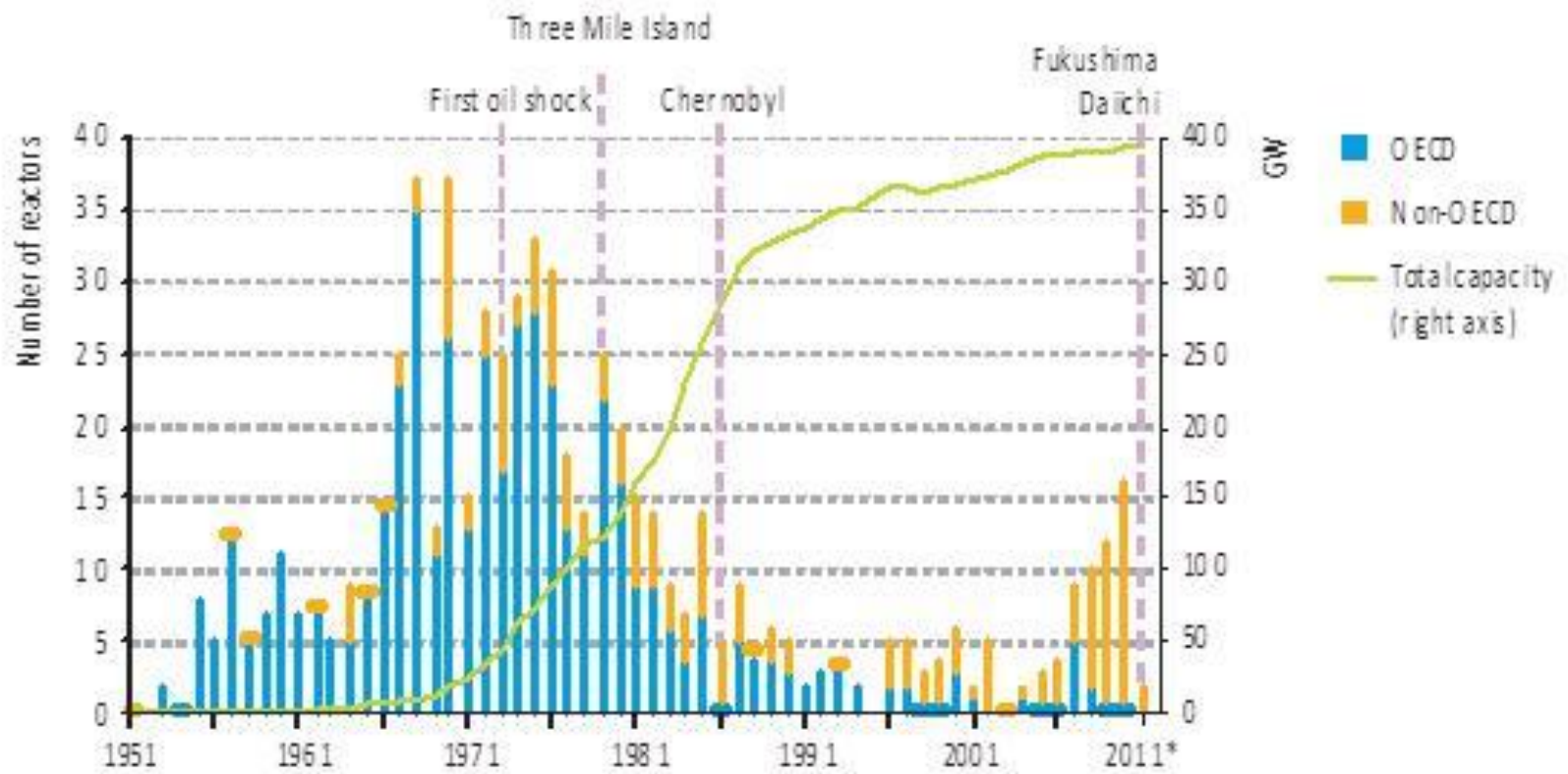


人災事故の リスクの比較

1974年WASH1400
ラスムッセン報告

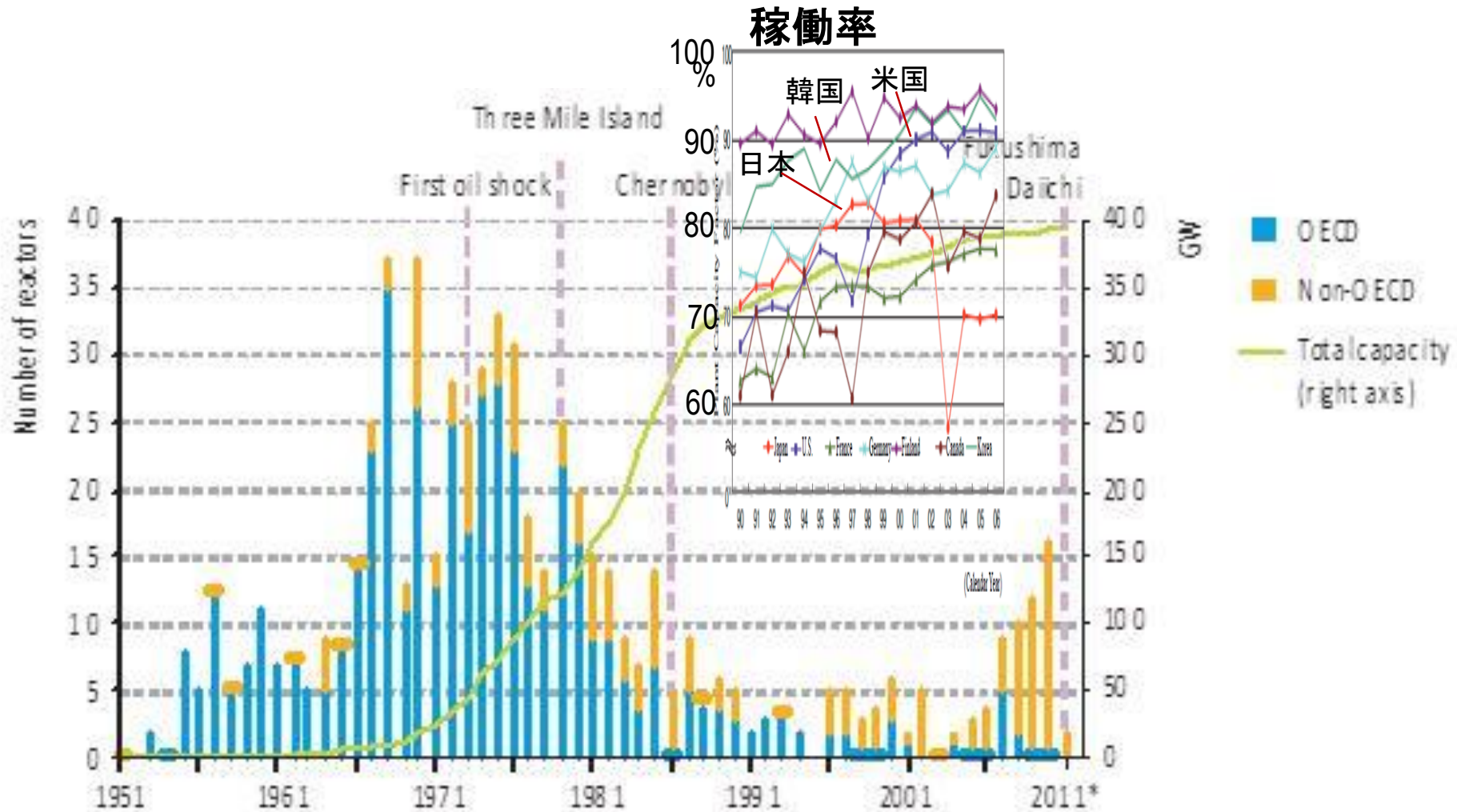


完成原子炉数と全出力



*Data as of 31 Aug 2011.

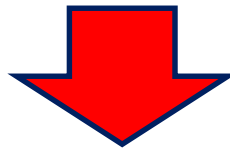
完成原子炉数と全出力



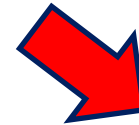
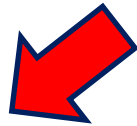
*Data as of 31 Aug 2011.

福島第一原子炉事故

廃炉



対応



軽水炉の安全性を高める

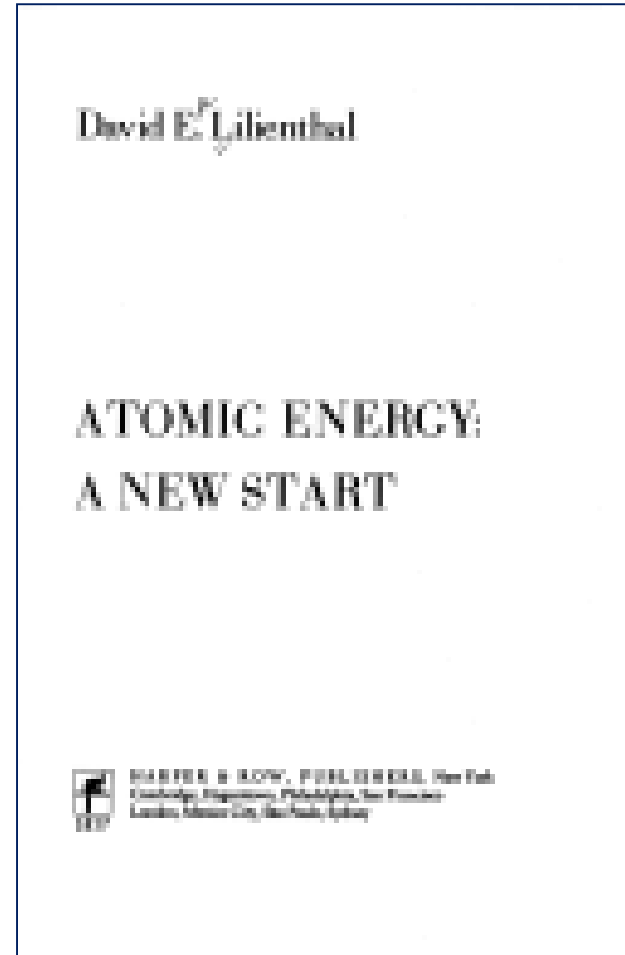
ストレステスト
安全設備、設計変更
システム改善

固有安全炉を使う

革新炉
小型炉

David E. Lilienthal

1st Chair of the AEC (1946 – 1950)



1980
9

固有安全炉

固有安全炉とは全ての安全機能が人間の操作や動的な機器に頼らず、物理的法則によっている原子炉である。受動的な安全炉とも呼ばれる。

安全機能:	止める(未臨界にする)	能動的→受動的
	冷やす(崩壊熱)	能動的→受動的
	閉じ込める(放射性物質)	より確実に

特長: EPZが原子炉敷地内に?

Inherently Safe Reactors and a Second Nuclear Era

Alvin M. Weinberg and Irving Spiwak

David Lilienthal, in his book *Atomic Energy, A New Start (1)*, was among the first to call upon nuclear technologists to design a reactor that was inherently safe. He saw such a device as being necessary for a new start in atomic energy. Without such a forgiving reactor, Lilienthal doubted t

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thal to call for an inherently safe reactor, but that the fundamental characteristics of the fission process, in particular the afterheat, made such a goal all but unattainable (2).

Nevertheless, in May of 1980, the Institute for Energy Analysis, under the

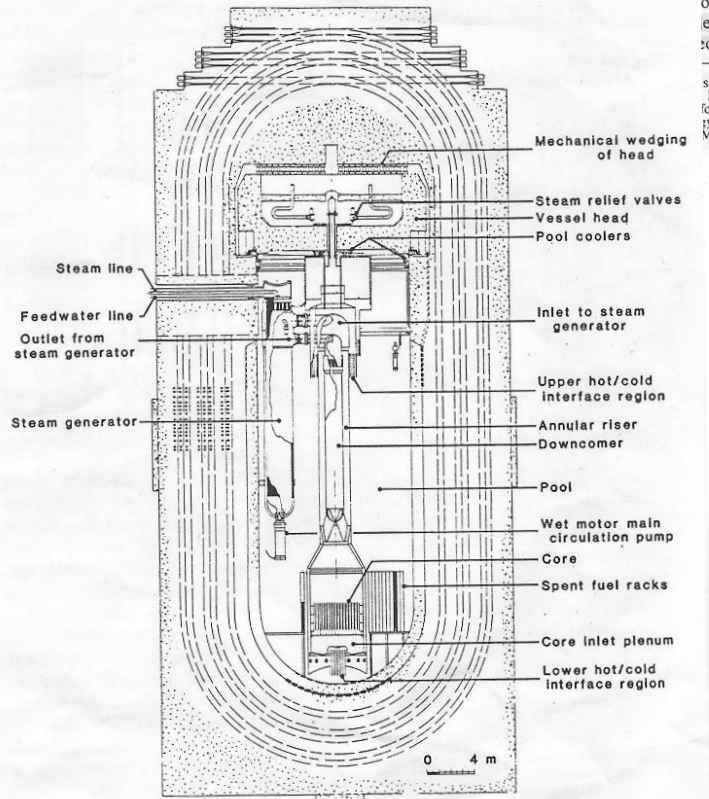
P. Cohen of Westinghouse, J. Dietrich of Combustion Engineering, M. Edlund of Babcock and Wilcox, P. Fortescue of General Atomic, K. Davis, then of Bechtel and later deputy secretary of energy, H. Kendrick of Department of Energy, U. Gat of Oak Ridge National Laboratory, and H. G. MacPherson, J. A. Lane, E. P. Epler, M. W. Firebaugh, and the authors, associated with the Institute itself.

We concluded that a serious study of more forgiving, or perhaps even inherently safe, reactors was a good idea, but the study would have to begin by assessing the safety of existing light-water reactors and of incremental improvements to light-water reactors (3). Most of the participants in the workshop believed that such a reexamination would confirm

the view that light-water reactors were as safe as any reactors that could be economically built with them. The workshop was led by Andrew W. Mellon and the Institute for En-

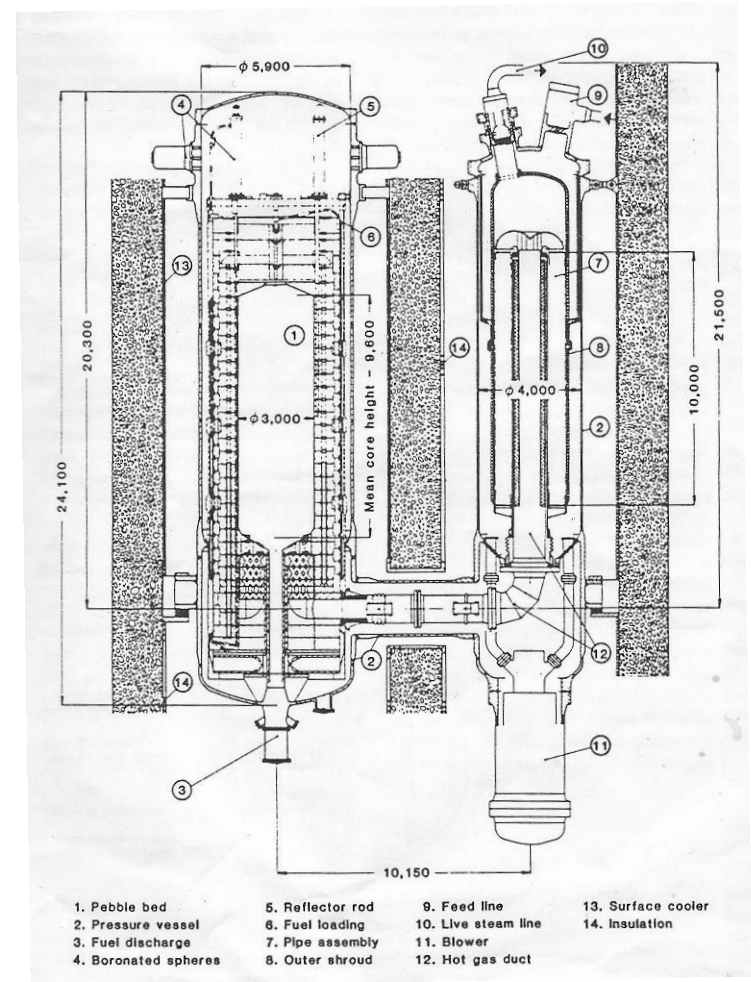
Energy Analysis, Oak Ridge National Laboratory, Tennessee, was the director of the Institute for Energy Analysis, Oak Ridge National Laboratory, Tennessee, formerly section head in the Energy Division, Oak Ridge National Laboratory, Tennessee, Mellon Fellow at the Institute

SCIENCE, VOL. 224



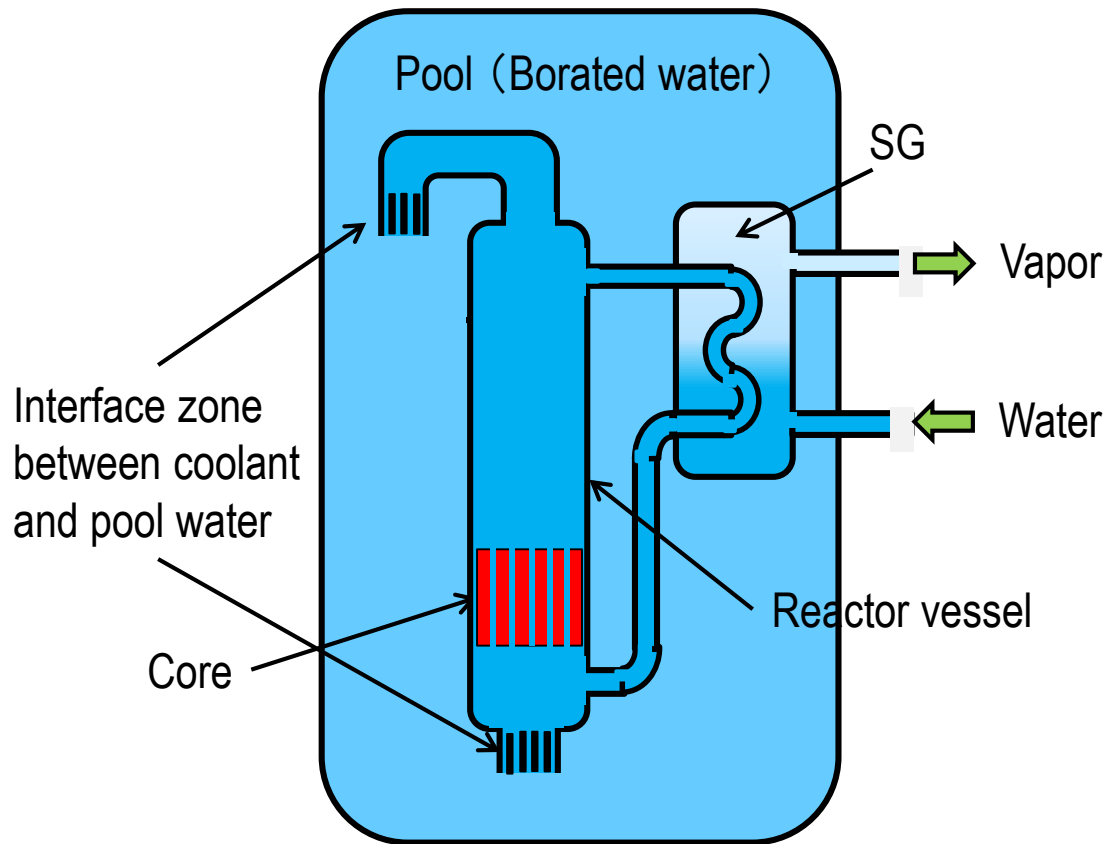
PIUS reactor

SCIENCE, Vol. 224: 29 June 1984

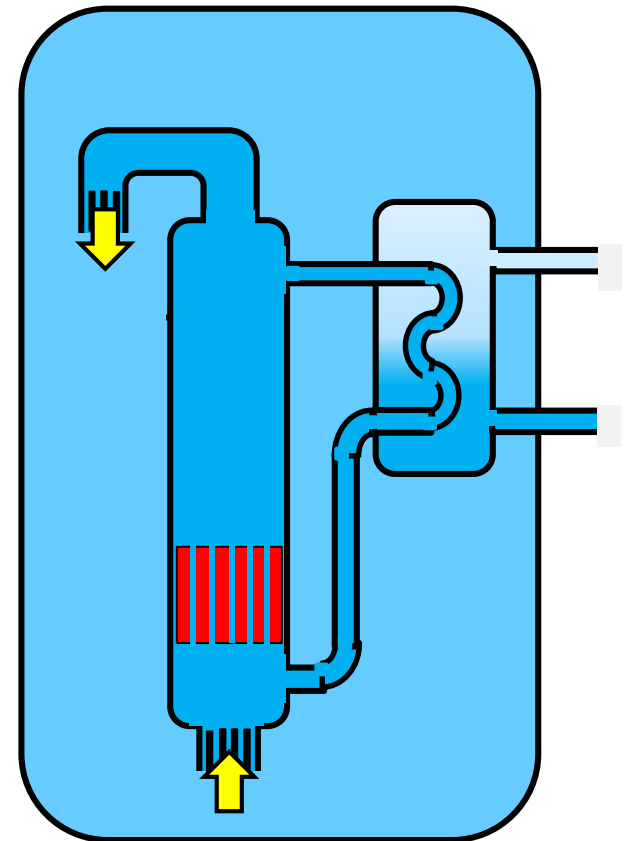


Modular HTR reactor

PIUS Reactor



normal operation



accident

Inherently Safe Reactors and a Second Nuclear Era

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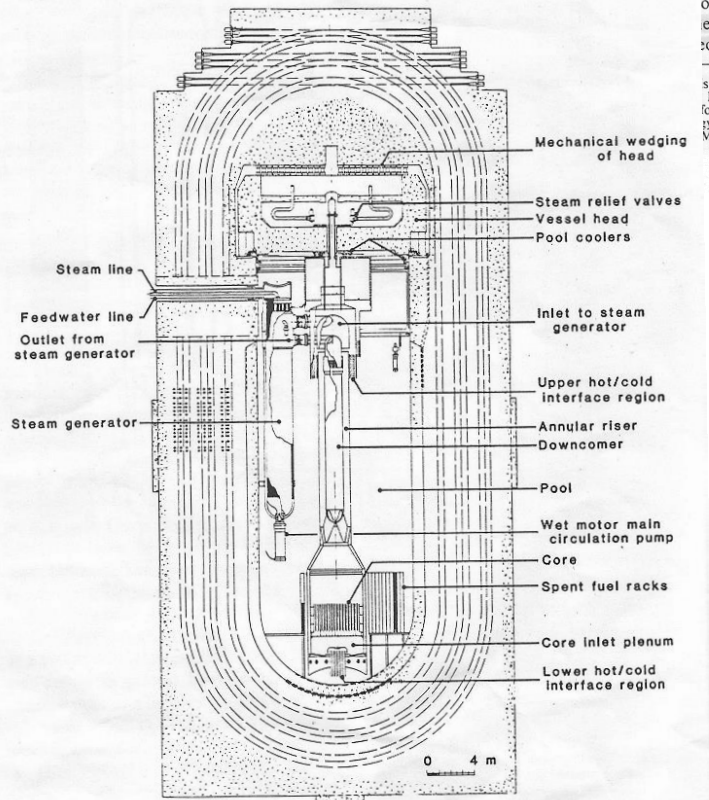
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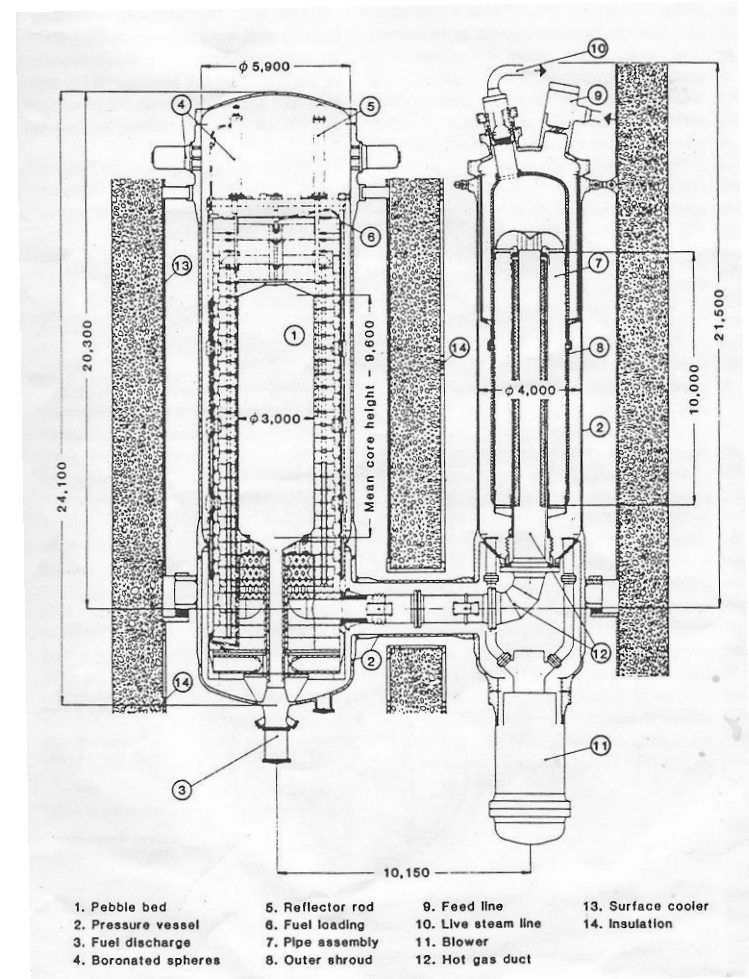
Energy Research was the director of the Institute for Energy Research at Oak Ridge Associated Universities, formerly section head in the Energy Division, Oak Ridge National Laboratory, and Mellon Fellow at the Institute

SCIENCE, VOL. 224



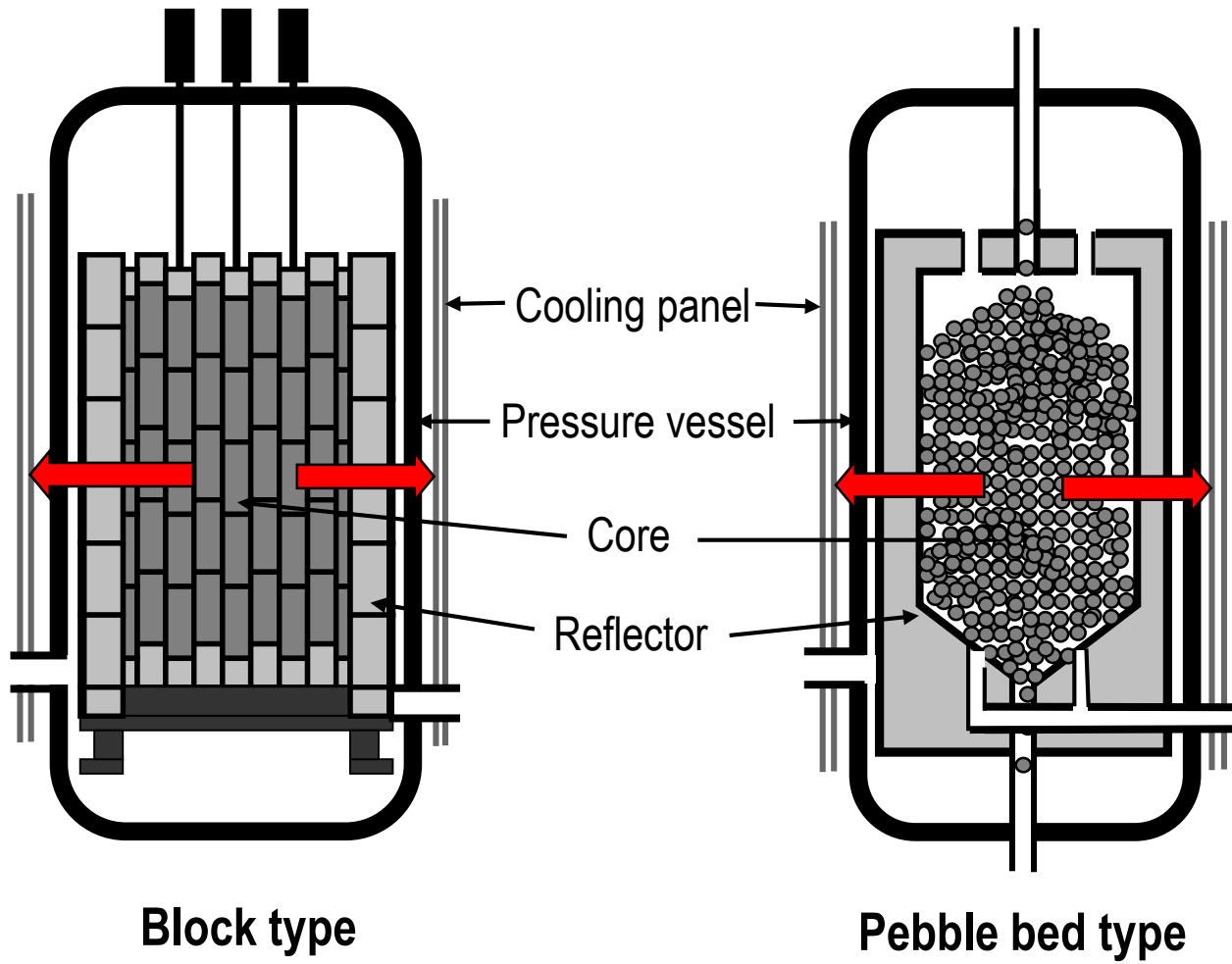
PIUS reactor

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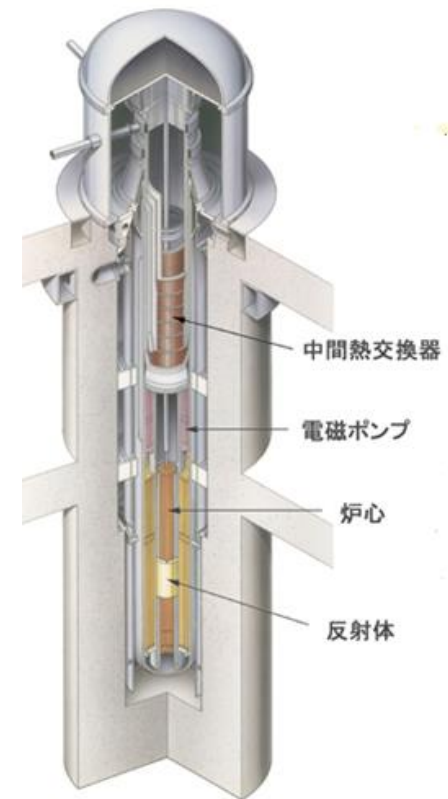
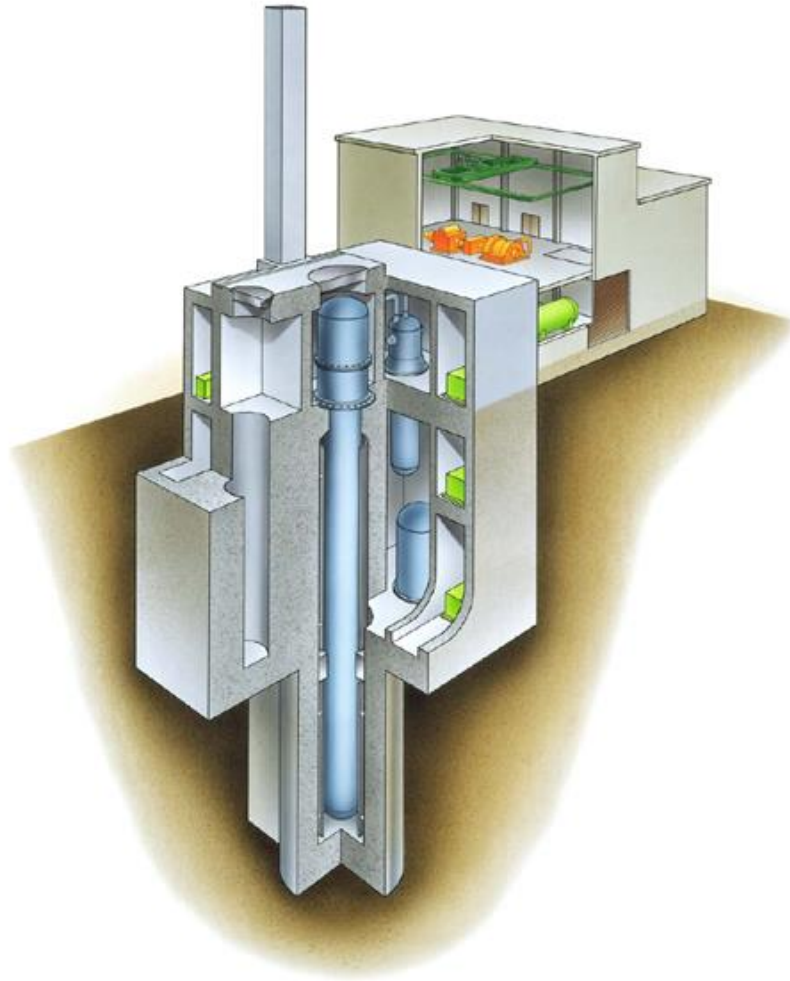
Modular HTR reactor

Modular HTR Reactor



← Heat flow at accident

4S (Super Safe, Small and Simple)



Main Design Parameters

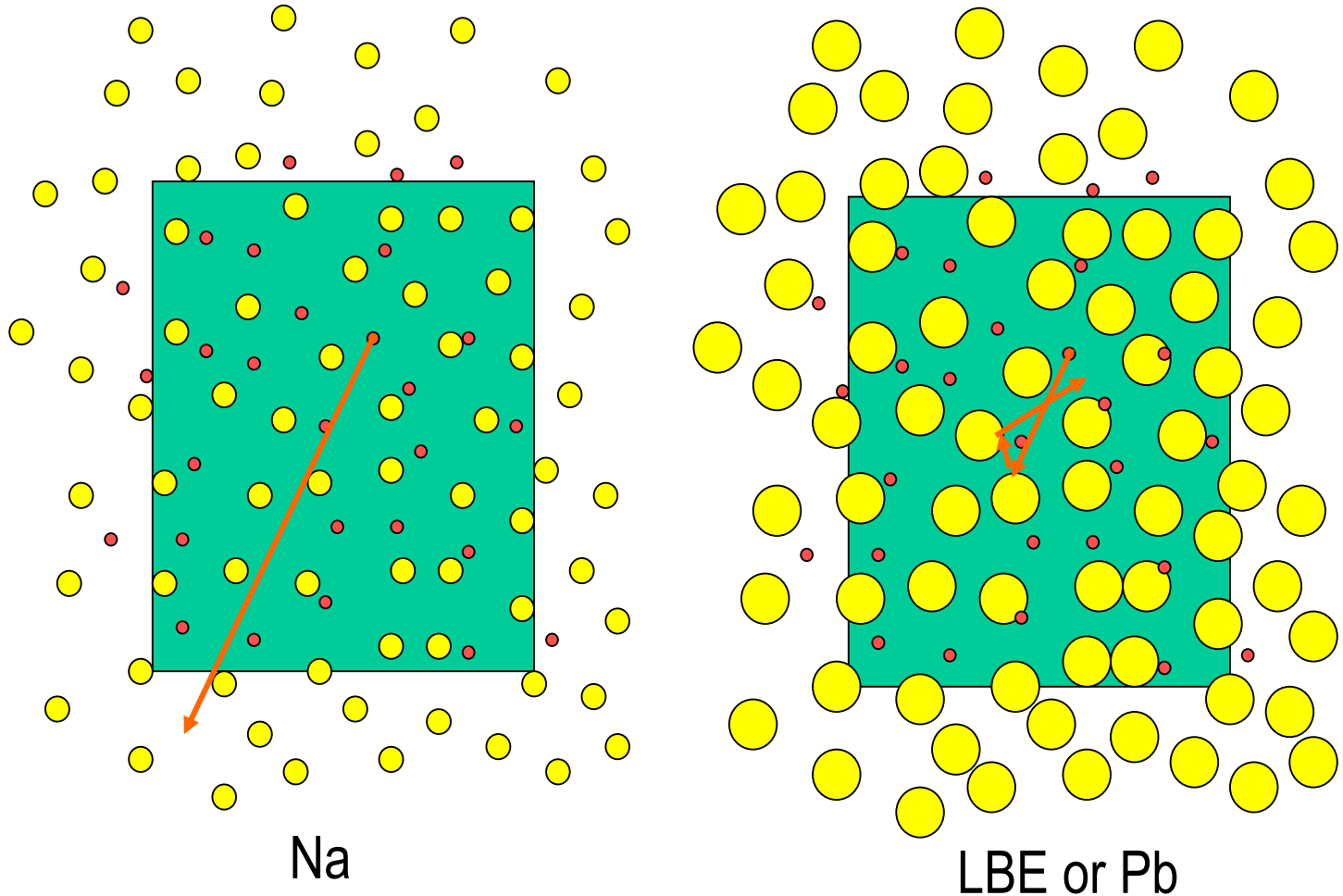


LSPR

LBE-Cooled **L**ong-Life
Safe **S**imple **S**mall
Portable **P**roliferation-
Resistant **R**eactor

Reactor Thermal Output	150 MWt
Reactor Electric Output	53 MWe
Reactor Outlet Temperature	510 °C
Reactor Inlet Temperature	360 °C
Reactor Vessel Diameter	5.2 m
Reactor Vessel Height	15.2 m
Core Barrel Diameter	3.4 m
Type of Steam Generator	Serpentine Tube
No. of Steam Generators	2 unit
Type of Pump	Centrifuge Pump
No. of Circulating Pump	2 unit
Total Pressure Drop	0.7 kg/cm ²
Pb-Bi Coolant Flow Rate	12300 ton/hr
Pb-Bi Coolant Core Velocity	0.9 m/s
Feed Water Temperature	210 °C
Feed Water Flow Rate	294 ton/hr
SG Outlet Steam Temperature	280 °C
SG Outlet Steam Pressure	6.47 MPa
Turbine Efficiency	35 %

中性子の漏えい



散乱断面積の大きいLBEやPbは中性子の閉じ込め性能が高い

- 1) 閉じ込め性能が高い冷却材は小型炉の設計が容易になる。
- 2) 閉じ込め性能が高い冷却材が無くなると、反応度が大きく下がり安全性が高まる。

小型炉の安全性

安全機能

止める（未臨界にする）

反応度係数がより負←中性子の漏れの効果が大
受動的原子炉停止が容易に

冷やす（崩壊熱）

炉心表面からの熱放出大←炉心半径小
受動的崩壊熱除去が容易に

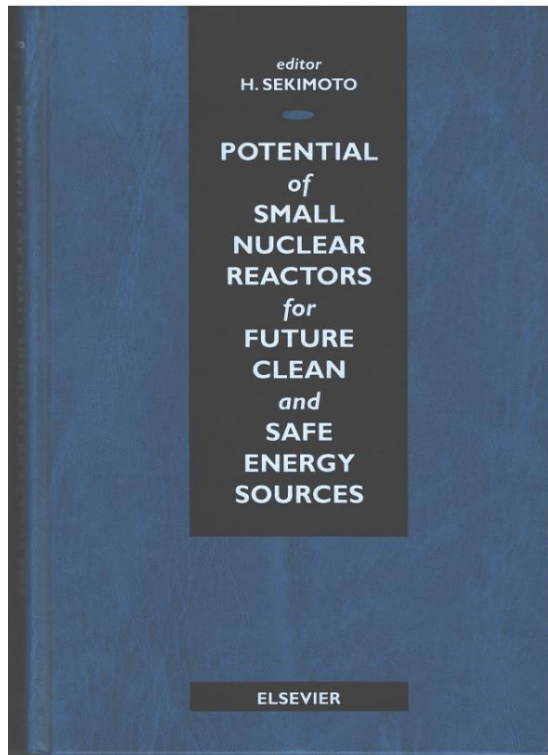
閉じ込める（放射性物質）

放射能の閉じ込め量が小
漏れの確率が小←表面積小
地下立地が容易

小型炉国際会議

SR/TIT

International Specialists' Meeting on
Potential of Small Nuclear Reactors for
Future Clean and Safe Energy Sources
Tokyo, Japan, 23-25 October, 1991



Water Cooled Reactors

SPWR, TRIGA-PS, DHR, MRX, DRX

Gas Cooled Reactors

MHTGR, Peu-a-Peu, CNPS

Fast Reactors

IFR, 4S, LSPR

Other Reactors

(mini)FUJI, Fluidized-Bed

小型炉リスト

Name	Capacity	Type*	Developer
CAREM	27-100 MWe	PWR	CNEA + INVAP, Argentina
MRX	30-100 MWe	PWR	JAERI, Japan
KLT-40S	35 MWe	PWR	OKBM, Russia
NuScale	45 MWe	PWR	NuScale Power + Fluor, USA
Flexblue	50-250 MWe	PWR	Areva TA, France
SMART	100 MWe	PWR	KAERI, South Korea
ACP100	100 MWe	PWR	CNNC + Guodian, China
NP-300	100-300 MWe	PWR	Areva TA, France
IRIS	100-335 MWe	PWR	Westinghouse-led, international
mPower	150-180 MWe	PWR	B&W + Bechtel, USA
SMR-160	160 MWe	PWR	Holtec, USA
Westinghouse SMR	225 MWe	PWR	Westinghouse, USA
VK-300	300 MWe	BWR	Atomenergoproekt, Russia
PBMR	165 MWe	HTR	Escom, South Africa, et al.
HTR-PM	2x100 MWe	HTR	INET + HSNPC, China
SC-HTGR (Antares)	250 MWe	HTR	Areva, France
GT-MHR	285 MWe	HTR	GA + Minatom, USA-Russia
4S	10-50 MWe	FNR	Toshiba, Japan
SVBR	10-100 MWe	FNR	AKME (Rosatom), Russia
Hyperion Power Module	25 MWe	FNR	Hyperion Pwr Gen, USA
LSPR, PBWFR	50-150 MWe	FNR	TokyoTech, Japan
ALFRED	120-600 MWe	FNR	Ansaldo, Italy
EM ²	240 MWe	FNR	GA, USA
BREST	300 MWe	FNR	RDIPe, Russia
S-PRISM	311 MWe	FNR	GE-Hitachi, USA
PB-AHTR	410 MWe	FHR	MIT + UCB + UWM, USA
FUJI, miniFUJI	10, 100-200 MWe	MSR	IThEMS, Japan-Russia-USA
IMSR	45 MWe	MSR	Terrestrial Energy, USA

* PWR: pressurized water reactor

BWR: boiling water reactor

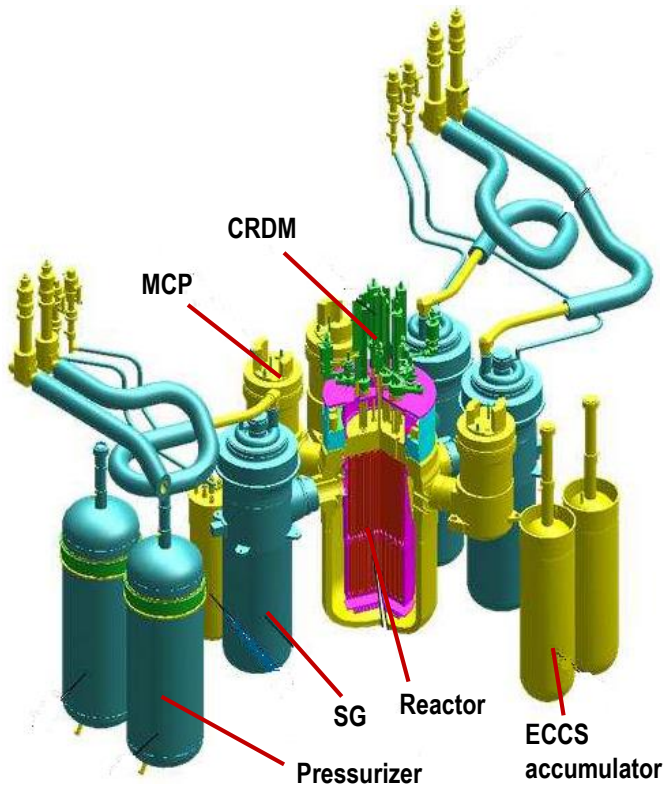
HTR: high-temperature reactor

FNR: fast neutron reactor

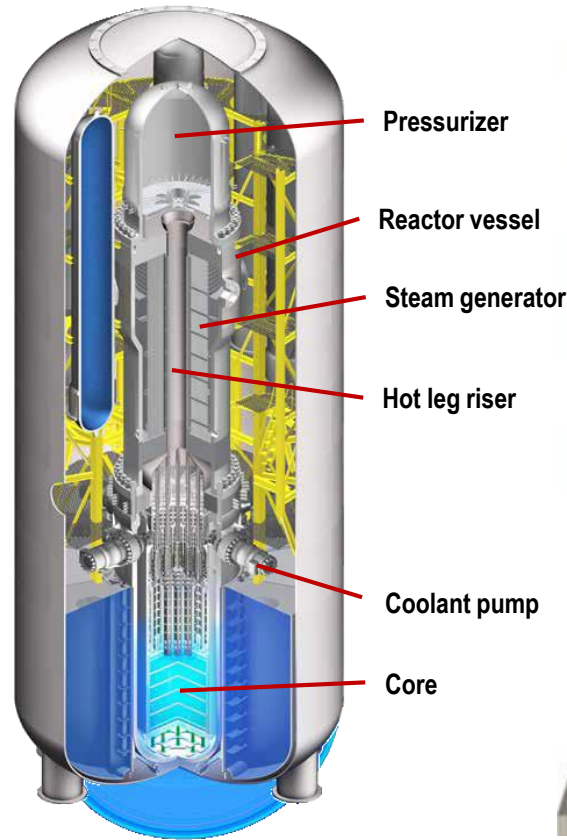
FHR: Flibe cooled HTR

MSR: molten salt reactor

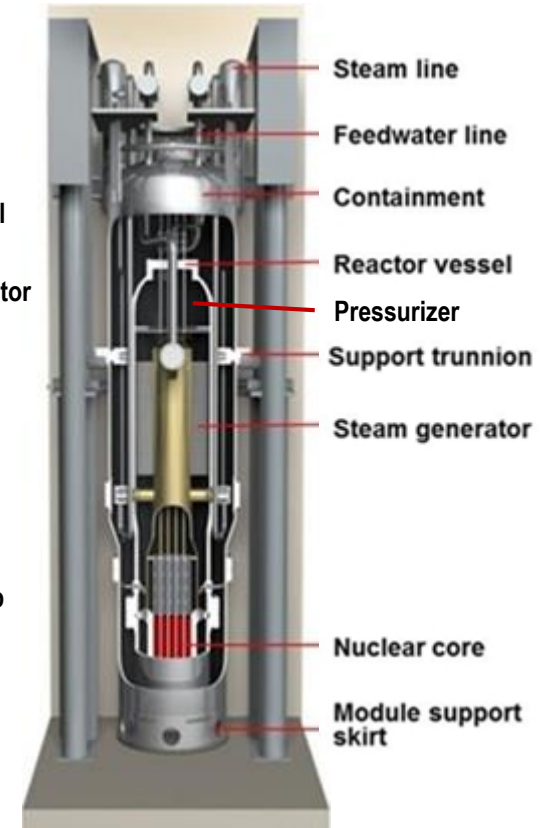
Some of the Present Small LWRs



KLT-40S



Westinghouse SMR

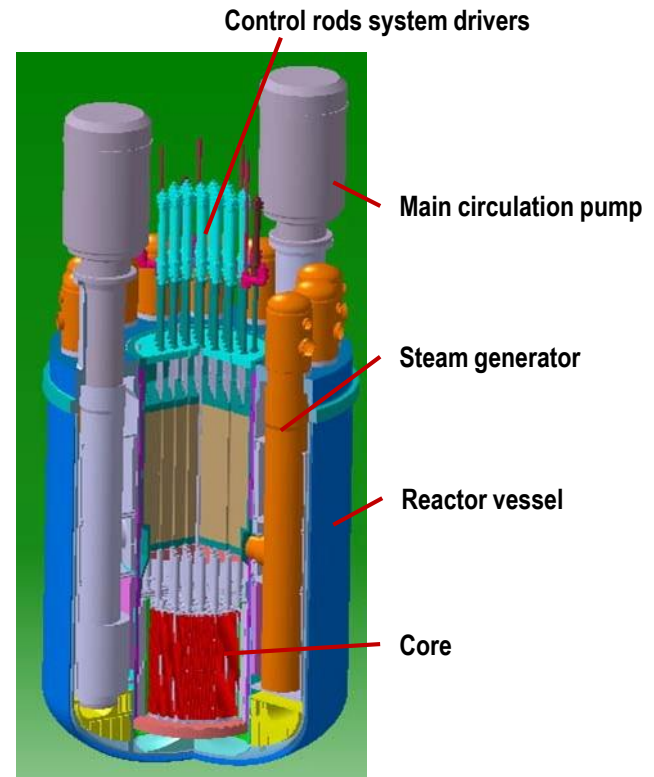


NuScale

Some of the Present Small FR



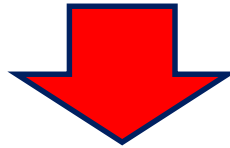
EM²



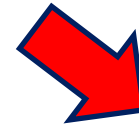
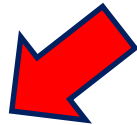
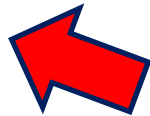
SVBR

福島第一原子炉事故

廃炉



対応



軽水炉の安全性を高める

ストレステスト
安全設備、設計変更
システム改善

固有安全炉を使う

革新炉
小型炉

ご清聴
ありがとうございました